#### NRC FORM 618 U.S. NUCLEAR REGULATORY COMMISSION (8-2000) 10 CFR 71 CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES b. REVISION NUMBER d. PACKAGE IDENTIFICATION NUMBER a. CERTIFICATE NUMBER c. DOCKET NUMBER PAGE PAGES 71-9288 USA/9288/B(U)F-96 OF 9288 10 1 6

## 2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION
  - a. ISSUED TO (Name and Address)
     Columbiana Hi Tech, LLC
     1802 Fairfax Road
     Greensboro, NC 27407

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION
Columbiana Hi Tech, LLC, consolidated application
dated February 27, 2006, as supplemented

### 4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

CLEAR

5.

- (a) Packaging
  - (1) Model No.: CHT-OP-TU
  - (2) Description

A shipping container for uranium oxide pellets, powder, and uranium-bearing materials. The package is roughly cubical and is approximately 45-inches x 45-inches x 62-inches high. The package has four internal sleeves in which Oxide Vessels are inserted.

The outer shell of the package is constructed of 11-gauge mild or stainless steel and the space between the outer shell and the sleeves are filled with fire retardant, closed cell phenolic or polyurethane foam.

The sleeves are constructed of 11-gauge mild or stainless steel with an inner diameter of 10-1/4 inches. The sleeves are closed with twelve 1/2-inch-diameter bolts using an outer lid assembly on a 1/16-inch-thick neoprene or silicone gasket. The outer lid assembly is filled with fire-retardant, closed cell phenolic or polyurethane foam.

The Oxide Vessel is constructed of series 300 stainless steel, with an inner diameter of either 6, 7.5, or 8 inches. The Oxide Vessel is closed by eight 1/2-inch-diameter bolts on a 5/8-inch-thick stainless steel lid with a double O-ring seal. The O-ring seal material is either

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|---|--------------------|------------------|----------------------------------|------|----|-------|--|--|--|--|--|
| CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES |                    |                  |                                  |      |    |       |  |  |  |  |  |
| <sup>1</sup> a. CERTIFICATE NUMBER                          | b. REVISION NUMBER | c. DOCKET NUMBER | d. PACKAGE IDENTIFICATION NUMBER | PAGE |    | PAGES |  |  |  |  |  |
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#### 5.(a) (2) Description (Continued)

silicon rubber, fluorosilicon or fluorocarbon (viton). A pellet shipping assembly is used within the Oxide Vessel for certain shipments.

The approximate dimensions and weights of the package are as follows:

Sleeve inside diameter 10 1/4-inches Oxide Vessel inside diameter 6, 7.5, or 8 inches Oxide Vessel inside height 40 3/4-inches Overall package dimensions width 45 inches length 45 inches height 62 inches Maximum contents weight 402 pounds per Oxide Vessel Maximum empty transport weight including four empty Oxide Vessels 2576 pounds Maximum loaded package weight

(with four filled Oxide Vessels) 3757 pounds

#### (3) Drawings

The packaging is constructed and assembled in accordance with Columbiana Hi Tech Drawing Nos.:

OP-TU-SAR, Rev. 12, Sheets 1 of 2 and 2 of 2; OP-TU-A2, Rev. 12, Sheet 1 of 1; OP-TU-A3, Rev. 12, Sheet 1 of 1; OP-TU-A4, Rev. 12, Sheet 1 of 1; and, OPTU-V-AB1, Rev. 8, Sheets 1 of 2 and 2 of 2.

The Oxide Vessel Pellet Shipping Assembly is constructed and assembled in accordance with AREVA NP, Inc., Drawing No. 9046816, Rev. 1.

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## 5.(b) Contents

(1) Type and form of material

Uranium-bearing compounds in solid form, heterogenous or homogenous (i.e., pellets and powder). The contents may include up to 1000 grams of polyethylene or other plastics as packaging, waste or impurities per Oxide Vessel (4000 grams per package), provided that: (1) the total water equivalent of the plastic is less than 1307 grams per Oxide Vessel (5228 grams per package); and, (2) the decay heat is less than 0.068 W/m³. Materials with a decay heat greater than 0.068 W/m³ may not be packaged using hydrogen bearing plastics, and may only use non-hydrogen bearing plastics such as Teflon<sup>TM</sup> (polytetrafluoroethylene or PTFE) or metallic containers. In addition, the contents are limited to:

- A. Unirradiated uranium oxide powder enriched to no more than 5.0 weight percent in the U-235 isotope.
- B. Unirradiated uranium oxide pellets or a mixture of pellets and powder enriched to no more than 5.0 weight percent in the U-235 isotope.
- C. Reprocessed uranium oxide powder enriched to no more than 5.0 weight percent in the U-235 isotope, with limits specified in Table 1.

Table 1: Allowable Content for Shipment of Reprocessed Uranium Oxide

| 300a                           | Maximum Content   |                |                 |  |  |  |
|--------------------------------|---|----------------|-----------------|--|--|--|
| Isotope                        | Type A  | Type B Level I | Type B Level II |  |  |  |
| U-232 (g/gU)                   | Mixtures of isotopes  | 2.00E-09       | 5.00E-09        |  |  |  |
| U-234 (g/gU)                   | shall be evaluated and designated as a Type   | 2.00E-03       | 2.00E-03        |  |  |  |
| U-235 (g/gU)                   | A quantity per 10 CFR Part 71 Appendix A. The maximum enrichment per package is 5 weight per cent <sup>235</sup> U. | 5.00E-02       | 5.00E-02        |  |  |  |
| U-236 (g/gU)                   |   | 2.50E-02       | 2.50E-01        |  |  |  |
| Np-237 (g/gU)                  |   | 1.66E-06       | 5.00E-03        |  |  |  |
| Pu-238 (g/gU)                  |   | 6.20E-11       | 4.00E-08        |  |  |  |
| Pu-239 (g/gU)                  |   | 3.04E-09       | 3.04E-09        |  |  |  |
| Pu-240 (g/gU)                  |   | 3.04E-09       | 6.00E-09        |  |  |  |
| Gamma Emitters<br>(MeV-Bq/kgU) |   | 6.38E+05       | 1.91E+06        |  |  |  |

| NRC FORM 618 U.S. NUCLEAR REGULATORY COMMISSION (8-2000) 10 CFR 71 CERTIFICATE OF COMPLIANCE |                    |                  |                                  |      |    |       |  |  |  |  |
|--|--------------------|------------------|----------------------------------|------|----|-------|--|--|--|--|
| FOR RADIOACTIVE MATERIAL PACKAGES  |                    |                  |                                  |      |    |       |  |  |  |  |
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- 5. (b) (1) Type and Form of Material (Continued)
  - D. Reprocessed uranium oxide pellets or a mixture of pellets and powder enriched to no more than 5.0 weight percent in the U-235 isotope, with the limits specified in Table 1.
  - E. Homogeneous (powder or crystalline form) uranium-bearing materials enriched to 5.0 weight percent in the U-235 isotope in the form of solids, or solidified or dewatered materials.

Uranium compounds must have a ratio of non-fissile atoms to uranium atoms greater than two (2) and the density of these compounds is less than 10.96 g/cm $^3$  (density of UO $_2$ ). Material such as U-metal, U-metal alloys, or uranium hydrides (e.g., UH $_X$ ) may not be shipped. Uranium-bearing materials may include oxides, carbides, silicates or other compounds of uranium. Uranium-bearing materials may be moderated by graphite to any degree. Compounds may be mixed with other non-fissile materials with the exception of beryllium or hydrogenous material enriched in deuterium. Materials with a hydrogen density greater than water must be excluded, except for the allowance provided by Condition No. 5.(b)(1).

F. Heterogeneous (pellets or previously pelletized materials) uranium-bearing materials enriched to 5.0 weight percent in the U-235 isotope in the form of solids, or solidified or dewatered materials.

Uranium compounds must have a ratio of non-fissile atoms to uranium atoms greater than two (2) and the density of these compounds is less than  $10.96 \text{ g/cm}^3$  (density of  $UO_2$ ). Material such as U-metal, U-metal alloys, or uranium hydrides (e.g.,  $UH_X$ ) may not be shipped. Uranium-bearing materials may include oxides, carbides, silicates or other compounds of uranium. Uranium-bearing materials may be moderated by graphite to any degree. Compounds may be mixed with other non-fissile materials with the exception of beryllium or hydrogenous material enriched in deuterium. Materials with a hydrogen density greater than water must be excluded, except for the allowance provided by Condition No. 5.(b)(1).

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5. (b)(2) Maximum quantity of material per package

The maximum allowable contents heat generation rate is 1.0 BTU/hr/ft³ (10.3 W/m³). The maximum weight of contents, including the uranium compounds and all packaging materials within the Oxide Vessel, is 402 pounds per 8-inch, 7.5-inch, or 6-inch diameter Oxide Vessel, and a maximum of 1608 pounds per package.

For contents described in Condition Nos. 5(b)(1)(B), 5.(b)(1)(D), and 5.(b)(1)(F), the Oxide Vessel Pellet Shipping Assembly, as described in Condition No. 5(a)(3), must be used within the 8-inch diameter Oxide Vessel. The Oxide Vessel Pellet Shipping Assembly is not required when using the 7.5-inch, or 6-inch diameter Oxide Vessel.

- (c) Criticality Safety Index
- 2.0
- 6. In addition to the requirements of Subpart G of 10 CFR Part 71:
  - (a) The package must be prepared for shipment and operated in accordance with the Operating Procedures in Section 7 of the application, as supplemented.
  - (b) Each packaging must be acceptance tested and maintained in accordance with the Acceptance Tests and Maintenance Program in Section 8 of the application, as supplemented.
- 7. Transport by air of fissile material is not authorized.
- 8. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.
- 9. Revision No. 9 of this certificate may be used until April 30, 2016
- 10. Expiration date: April 30, 2020.

U.S. NUCLEAR REGULATORY COMMISSION NRC FORM 618 (8-2000) 10 CFR 71 **CERTIFICATE OF COMPLIANCE** FOR RADIOACTIVE MATERIAL PACKAGES d PACKAGE IDENTIFICATION NUMBER a. CERTIFICATE NUMBER b. REVISION NUMBER c. DOCKET NUMBER PAGE PAGES OF USA/9288/B(U)F-96 6 9288 10 71-9288 6

# **REFERENCES**

Columbiana Hi Tech, LLC, consolidated application dated February 27, 2006.

Supplements dated: April 10, 2006; July 17 and August 29, 2007; January 18, 2010; January 28, 2015.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

B. Jennifer Davis, Acting Chief Spent Fuel Licensing Branch Division of Spent Fuel Management Office of Nuclear Material Safety and Safeguards

Date: April 2, 2015



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION REPORT
Docket No. 71-9288
Model No. CHT-OP-TU Package
Certificate of Compliance No. 9288
Revision No. 10

## **SUMMARY**

By the application dated January 28, 2015 (Agencywide Documents Access and Management System Accession Number No. ML15069A261), Columbiana Hi Tech, LLC (certificate holder) requested renewal of Certificate of Compliance (CoC) No. 9288 for the Model No. CHT-OP-TU package. The certificate holder did not request any changes to the package design or the safety analysis report. The certificate has been renewed for an additional five year term expiring on April 30, 2020.

## **EVALUATION**

By letter dated January 28, 2015, Columbiana Hi Tech, LLC, requested renewal of CoC No. 9288, for the Model No. CHT-OP-TU package. Columbiana Hi Tech, LLC, did not request any changes to the package design, authorized contents, operating procedures, acceptance tests, or maintenance program of the package.

Columbiana Hi Tech, LLC, described in the letter that the consolidated application for renewal of the existing CoC submitted incorporated supplements as referenced in the existing certificate into safety analysis report (SAR) Revision 8 dated September 27, 2007, but staff found out it had not been submitted. Columbiana Hi Tech, LLC, agreed with the staff's finding, and made it clear that the company will not request any changes to the references of the existing CoC.

The staff reviewed the documents referenced in the certificate and determined that the documentation was available and complete. The staff also reviewed the operating and maintenance procedures for the package and found them to be adequate.

The following changes have been made to the certificate:

Condition No. 9 was modified to authorize the use of the previous revision of the certificate for a period of approximately one year.

Condition No. 10 was modified to reflect the new expiration date to April 30, 2020.

# CONCLUSION

The certificate has been renewed for a five year term that expires on April 30, 2020. These changes do not affect the ability of the package to meet the requirements of 10 CFR Part 71.

Issued with CoC No. 9288, Revision No. 10, on April 2, 2015.